



Calibration of a Gamma-Ray Spectrometer using Electronic Spreadsheet Package and Dedicated Spectral Analysis Software

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ABSTRACT

As a first step after installation of new software and hardware in the Neutron Activation Analysis (NAA) laboratories of Nigeria Research Reactor-1 (NIRR-1) the calibration of the spectrometer was carried out before any spectral analysis could be done. Both electronic spreadsheet and dedicated spectral analysis software (k_0 -IAEA) were employed and the results obtained from both methods were compared in this study. The efficiency curves were established from measurement and interpretation of several spectra from nine standard gamma-ray calibration sources (Na-22, Mn-54, Co-57, Co-60, Y-88, Cs-137, Eu-152, Ra-226, Am-241) whose activities are known to better than $\pm 3\%$. The sets of values obtained for the full-energy peak detection efficiency from the two approaches are close at higher geometries with less than 10% variation.

Keywords: *Detector calibration; Efficiency; k_0 -standardization; k_0 -IAEA program; NIRR-1; Neutron activation analysis.*

I. INTRODUCTION

The search for a versatile Neutron Activation Analysis (NAA) Standardization method that meets the criteria of experimental simplicity, analytical accuracy and flexibility (with respect to activation and counting conditions) has led to acceptability and the adoption of the k_0 -based NAA standardization method for routine analysis in many laboratories all over the world [1]. These advantages come at a price of accurate characterization of the neutron flux parameters in all the irradiation channels and full calibration of the detectors [1-2]. For quantification of the radioactive products using the traditional relative method, samples and standard materials of known elemental concentrations are irradiated and counted under the same experimental conditions [3]. The physical parameters for the element of interest being identical in both the analyte and standard cancel out in the concentration calculations. This approach therefore eliminates the need for accurate determination of neutron fluxes and detector calibration. Furthermore, the k_0 -based NAA standardization method has no room for compromise in full determination of neutron fluxes and detector calibration. The interpretation of gamma ray spectra in terms of identifying the elements present, net peak area calculation to determine activity and subsequently the elemental concentration as well as detector calibrations could be performed by the "spreadsheet" approach using any commercial electronic spreadsheet package provided all the input parameters are known. But the complexity of most γ -ray spectra coupled with the computation requiring large data set such as neutron spectrum parameters, detector efficiencies, nuclear data, time parameters (irradiation, decay, counting times), sample-specific data (mass, matrix type), spectrum-specific data (peak energies and net peak areas, dead time) necessitated computerization and automation with special spectral analysis software. The acceptability and the computation demands of the k_0 -method led to the development

of several in-house computer programs in many different laboratories all over the world [4]. To ensure harmonization of the software used for the implementation of the k_0 -standardization of NAA and to further assist member States in their quality output from nuclear analytical laboratories, the International Atomic Energy Agency (IAEA) initiated the development of another spectral analysis software, the k_0 -IAEA. As a prerequisite for the implementation of the k_0 -NAA Standardized Method in the Nigeria Research Reactor-1 (NIRR-1), the characterization of the neutron flux parameters in the irradiation channels, development of the experimental protocols for the facilities and calibration of the detectors at different source-detector geometries have to be carried out. This characterization of the irradiation channels and the development of the experimental protocols have been described elsewhere [4-8]. This paper therefore focuses on the calibration of the detector. Proper, efficient and accurate calibration is critical to achieving accurate analytical results from k_0 -NAA Standardized laboratories by ensuring sample spectra are accurately interpreted. The calibration exercise allows the spectroscopic instrument to determine the composition of future samples, whose composition is completely unknown initially [9].

II. EXPERIMENTAL

A. Gamma ray spectrometer calibration using the k_0 -IAEA spectral analysis software involved configuration of the spectral acquisition system and the actual analysis and interpretation of various spectra from standard gamma ray calibration sources.

(i) Configuration of the spectral acquisition system

The High-Purity Germanium detector (HPGe) GEM 30195 was coupled to the integrated digital gamma-ray spectrometer hardware DSPEC jr 2.0 (Plate 1)

which is completely computer-controlled with the MAESTRO-32 multi-channel analyzer (MCA) emulation software. Complete computer control of the front end electronics means there are no knobs to turn or buttons to push, eliminating the possibility of accidental misadjustment. Precise adjustment of all front end electronics is accomplished via emulated controls - right on the screen. The DSPEC jr 2.0 is connected via a USB port to the host personal computer (Plate 2) which is used for configuring the hardware settings (such as high voltages, presets and amplifier gain) through the multichannel buffer (MCB) interface of the MAESTRO-32 software [10-14].



Plate 1. Digital Spectrometer



Plate 2. DSPEC jr 2.0 connected to desktop computer



Plate 3. GEM 30195 detector mounted on 30-liter liquid nitrogen dewar for cooling to cryogenic temperatures and surrounded by lead shield to reduce the background caused by sources other than the sample.

At the start of the day before acquisition of any spectrum, a routine energy calibration of the MAESTRO-32 spectrum acquisition software was carried out using a mixed energy gamma ray calibration sources consisting of three pure standard gamma ray calibration sources: ^{137}Cs , ^{60}Co and ^{88}Y . The sources were placed on a sample holder at about 10 cm from the detector end cap (Plate 3) and the spectrum acquired long enough to clearly identify the positions of the pure photopeaks in the spectrum. This usually takes about five minutes. The corresponding peaks were seen prominently at 661.6 KeV, 1173 keV, 1332 keV and 1836 keV for ^{137}Cs , ^{60}Co and ^{88}Y respectively. However, within this region, there was also a less significant impurity peak at 1460 keV due to ^{40}K . The calibration is carried out by replacing the channel number placeholders for these peaks on the horizontal axis of the spectrum plot with the well-known actual values of the expected peak energies from these calibration sources.

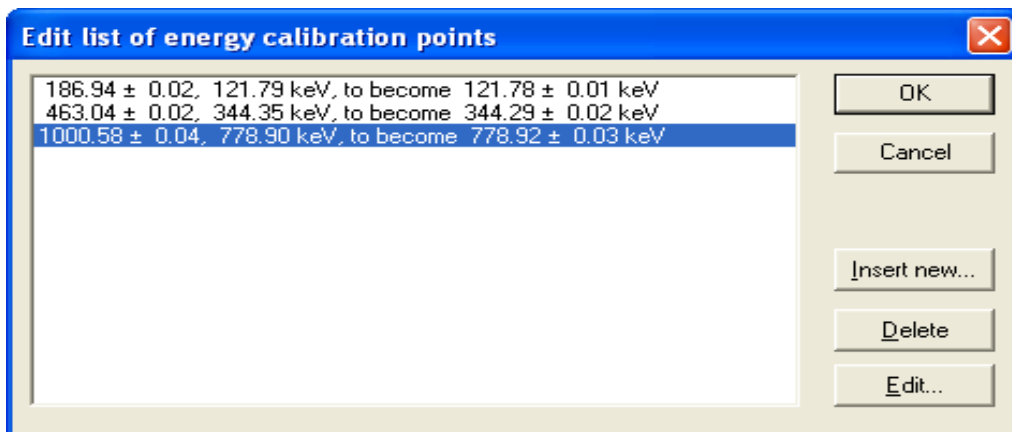


Fig. 1. Window of the menu "Spectrum analysis/Calibration/Energy calibration" with typical values of data entered for energy calibration with Eu-152 spectrum analysed with k_0 -IAEA program

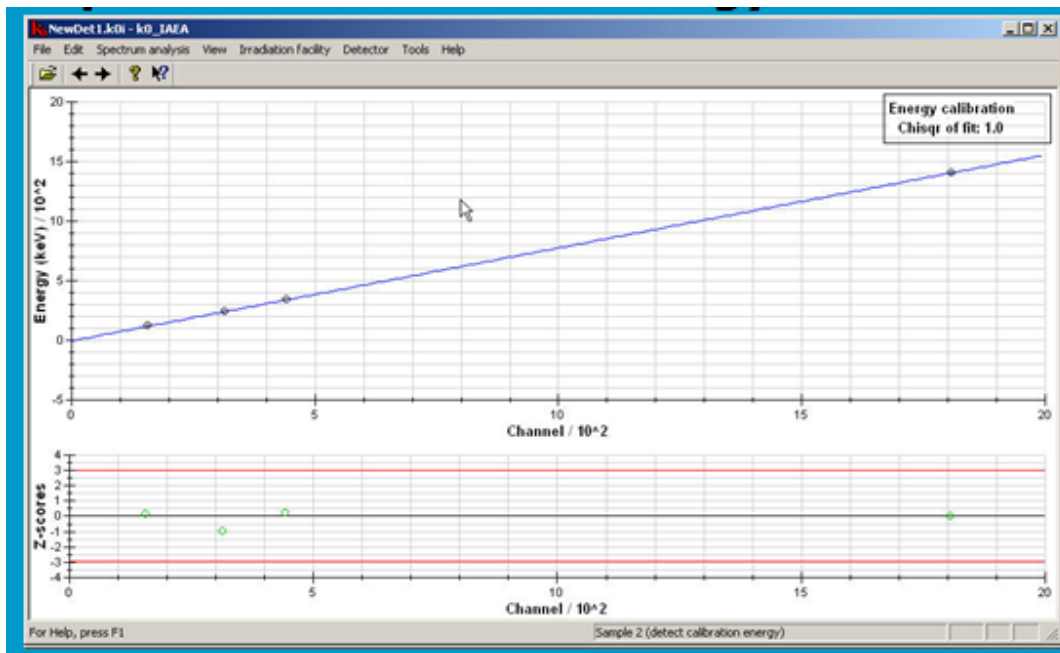


Fig. 2. Typical curve for energy calibration in k_0 -IAEA program.

(ii) Analysis and interpretation of various spectra from standard gamma ray calibration sources :

The Efficiency calibration establishes the relationship between the peak energy and the probability of the detector recording a count in the full energy peak. Establishing the efficiency curves from measurement involves using two spectra from two separate standard calibration sources whose activities are known accurately, the first being Cs-137 (a radionuclide that emit only one gamma-ray at a time), the second source is a multiple gamma ray emitter Eu-152 radionuclide calibration source. Interpreting the single-radionuclide spectrum produces the peak-to-total curve while interpreting the multi-gamma line spectrum produces a full-energy peak efficiency curve ^[14].

In this study, the farthest source-detector distance was used for the peak-to-total ratio calibration, where true-coincidence effects are negligible. The acquired spectrum for Cs-137 was loaded into the calibration series and interpreted using the command Edit → interpret selected sample. Since the calibration source certificate entered into the permanent database indicated that the nuclide was a single gamma line emitter, the program recognized that it would be calculating the peak-to-total ratio during the interpretation of its spectrum. Cs-137 standard source emits only one photo peak at 661.6 KeV energy ^[17]. The efficiency curve was then fitted, and viewed with the command View → fitted efficiency curve and finally stored by clicking on the menu command Detector → efficiency curve → store efficiency curves. Storing the fitted efficiency curve in the permanent database of the k_0 -IAEA program requires specifying the name of the detector and detector-source distance in the dialog box ^[12,15].

The last set of calibration samples interpreted were the full-energy peak efficiency spectra for all the various geometries to be used. Eu-152 calibration source was used for this study at 2

cm and 15 cm; the two optimum source-detector distances for the counting with the GEM 30195 detector. The acquired spectrum for each source-detector distance was loaded into the calibration series and interpreted using the command Edit → interpret selected sample. The calibration source certificate entered into the permanent database indicated the nuclide was a multiple gamma line emitter. Therefore, the program recognized that it was to calculate the full-energy peak efficiency curves during the interpretation of the spectrum. After fitting the curve to the points, the program converts the computed efficiencies from real non-point source to ideal mathematical point source geometry that have no mass, size and γ -ray self-absorption. "Ideal" calibration sources should be point sources ^[14]. It is this point source referenced efficiency data that is stored in the permanent database so that appropriate corrections is made for the geometrical conditions used in the actual analysis for measurement of bulky or extended samples, varying comparator shapes and sizes. This correction is determined by the matrix type, the geometry and the filling height ^[12,14,16]. After the computation, the numerical results can be viewed by clicking on the command View → numerical results. Furthermore, the fitted and the referenced efficiency curves can also be viewed using View → fitted efficiency curve and View → efficiency curve for the spectra respectively. The efficiency curve is finally stored by clicking on Detector → efficiency curve → store efficiency curves. The program prompts the user to specify the name of the detector and respective detector-source distances at this point. During routine analysis of real samples, the reference efficiency curve that is the closest match to the sample counting geometry is used. This is converted to the actual sample counting geometry using a semi-empirical approach based on the calculation of effective solid angles with a Monte Carlo method for which

knowledge of the geometrical parameters of the source-detector configuration is required. The detector's dimensions including the thickness of the dead layer and the distance from the germanium crystal to the housing were entered into the permanent database for this purpose. Generally, it is a good practice to perform full-energy peak detection calibration for all the geometries to be used for measurement during sample

analysis [14]. This improves accuracy because of much smaller correction factors for the conversion from reference geometry efficiency to actual sample geometry efficiency. [14,16] Efficiency calibrations were done for the two geometries used in this study over the operating range of approximately 100 - 3000 KeV. (Fig. 3 and Fig. 5)

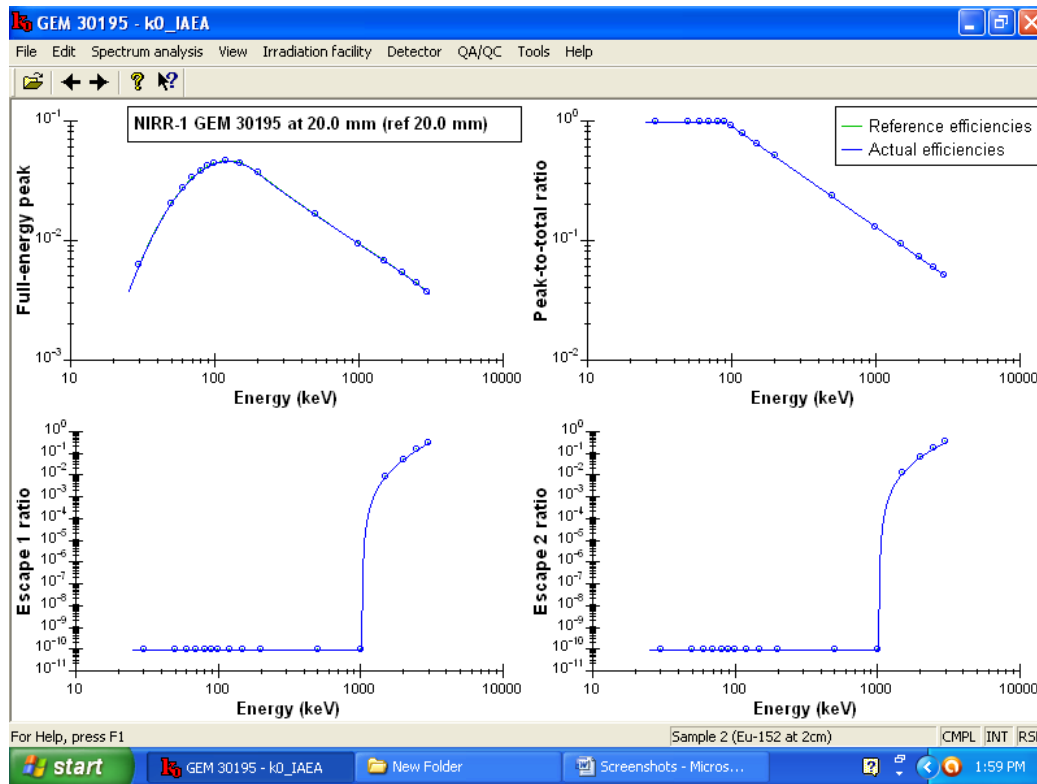


Fig. 3. Full energy efficiency and the peak-to-total curves for the GEM 30195 detector at 2 cm source-detector geometry.

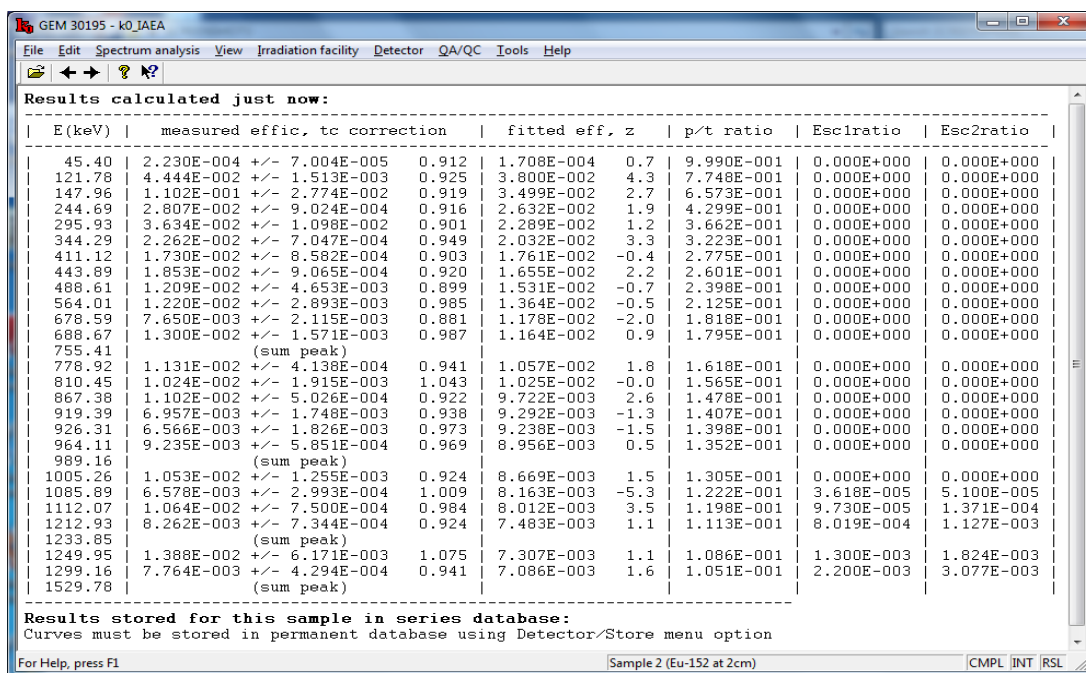


Fig. 4. Efficiency calibration datapoints for GEM 30195 at 2 cm source-detector geometry

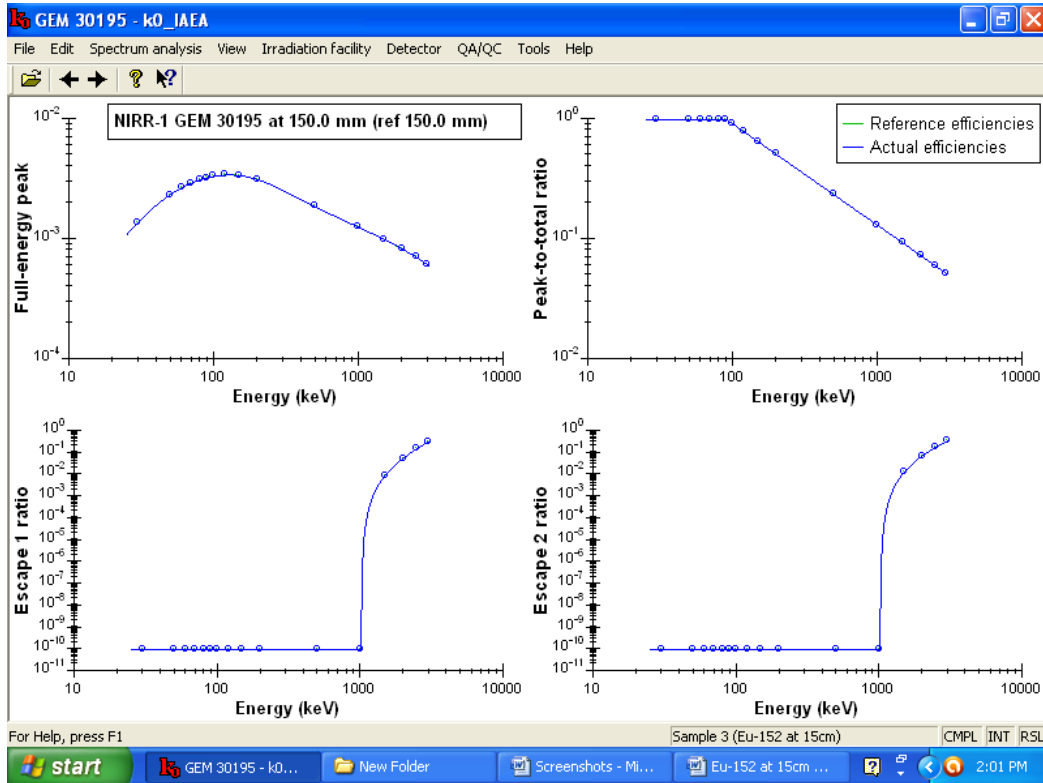


Fig. 5. Full energy efficiency and the peak-to-total curves for the GEM 30195 detector at 15 cm source-detector geometry

Results calculated just now:

E(keV)	measured effic, tc correction	fitted eff, z	p/t ratio	Esc1ratio	Esc2ratio		
121.78	3.453E-003 +/- 1.286E-004	0.989	3.415E-003	0.3	7.748E-001	0.000E+000	0.000E+000
244.69	2.802E-003 +/- 1.560E-004	0.989	2.818E-003	-0.1	4.299E-001	0.000E+000	0.000E+000
344.29	2.510E-003 +/- 1.912E-004	0.993	2.294E-003	1.1	3.223E-001	0.000E+000	0.000E+000
367.80	2.270E-003 +/- 3.932E-004	1.003	2.207E-003	0.2	3.048E-001	0.000E+000	0.000E+000
411.12	2.185E-003 +/- 2.891E-004	0.989	2.071E-003	0.4	2.775E-001	0.000E+000	0.000E+000
443.89	2.120E-003 +/- 1.322E-004	0.989	1.983E-003	1.0	2.601E-001	0.000E+000	0.000E+000
564.01	1.561E-003 +/- 6.064E-004	0.995	1.735E-003	-0.3	2.125E-001	0.000E+000	0.000E+000
688.67	1.597E-003 +/- 7.964E-004	0.998	1.557E-003	0.1	1.795E-001	0.000E+000	0.000E+000
719.42	1.414E-003 +/- 5.656E-004	0.994	1.521E-003	-0.2	1.730E-001	0.000E+000	0.000E+000
778.92	1.452E-003 +/- 7.058E-005	0.993	1.457E-003	-0.1	1.618E-001	0.000E+000	0.000E+000
867.38	1.488E-003 +/- 2.676E-004	0.992	1.376E-003	0.4	1.478E-001	0.000E+000	0.000E+000
919.39	1.281E-003 +/- 5.666E-004	0.993	1.335E-003	-0.1	1.407E-001	0.000E+000	0.000E+000
964.11	1.243E-003 +/- 5.375E-005	0.997	1.302E-003	-1.1	1.352E-001	0.000E+000	0.000E+000
1005.26	1.576E-003 +/- 2.699E-004	0.992	1.274E-003	1.1	1.305E-001	0.000E+000	0.000E+000
1085.89	1.243E-003 +/- 4.036E-004	1.000	1.223E-003	0.0	1.222E-001	3.618E-005	5.100E-005
1112.07	1.162E-003 +/- 4.905E-005	1.001	1.208E-003	-1.0	1.198E-001	9.730E-005	1.371E-004
1212.93	1.440E-003 +/- 1.849E-004	0.992	1.155E-003	1.5	1.113E-001	8.019E-004	1.127E-003
1292.78	1.344E-003 +/- 5.946E-004	1.002	1.117E-003	0.4	1.055E-001	2.067E-003	2.892E-003
1299.16	8.235E-004 +/- 8.467E-005	0.993	1.114E-003	-3.4	1.051E-001	2.200E-003	3.077E-003
1408.00	9.480E-004 +/- 6.026E-005	0.998	1.069E-003	-2.0	9.818E-002	5.269E-003	7.301E-003
1457.62	3.400E-003 +/- 4.372E-004	1.006	1.049E-003	5.4	9.535E-002	7.210E-003	9.938E-003
1528.12	8.869E-004 +/- 3.067E-004	1.128	1.024E-003	-0.4	9.163E-002	1.060E-002	1.449E-002

Results stored for this sample in series database:
Curves must be stored in permanent database using Detector/Store menu option

Fig. 6. Efficiency calibration datapoints for GEM 30195 at 15 cm source-detector geometry.

B. Full-energy peak efficiency curve fitting with electronic spreadsheet:

The detected activities of the radionuclides was measured with the detector and calculated using equation 1

$$Activity = \frac{Peak\ Net\ Area}{Live\ Time} \dots\dots\dots(1)$$

using the peak areas in the spectra acquired with MAESTRO-32 software. The actual current activities of the radionuclides were calculated using equation

$$A = A_0 e^{-\lambda t} \dots\dots\dots(2)$$

where A_0 = Initial activity of the radionuclide at the time of packaging, t is the duration of decay from time of packaging to time of measurement and λ = decay constant defined by equation 3.

$$\lambda = \frac{\log_e 2}{T_{\frac{1}{2}}} = \frac{0.693}{T_{\frac{1}{2}}} \dots\dots\dots(3)$$

where $T_{\frac{1}{2}}$ = half-life of the source.

The efficiency of each gamma line is calculated from equation 4

$$Efficiency = \frac{Experimental \text{ value of activity measured with detector}}{(Actual \text{ activity of calibration source}) \times (\text{branching ratio of } \gamma\text{-ray})} \dots(4)$$

This experiment was concerned with finding out how the value of the efficiency varies with the gamma-ray energy. Therefore, the graph of efficiency versus gamma-ray energy was plotted using the Microsoft Excel on a log-log scale as shown in Fig. 7. Since a non-linear relationship was obtained, a polynomial function of degree 3 was fitted to the experimental data points of the detector's efficiency curve. An interpolation of efficiency at any required gamma-ray

energy between two tabulated and plotted data points could then be calculated using the equation obtained from the trend line and regression analysis. The least square method was used as a criterion for determining the best fitting curve which is as close as possible to the experimental data. The correlation coefficient given by the electronic spreadsheet is a measure of how the two variables relate.

Table 1. Data from the certificates of the standard gamma ray calibration sources used.

Radionuclide	Activity	Uncertainty	Half-life	Packaging Date	Time	Material
Co-60	38400	1100	5.27 years	15-Jul-04	12:00	DKD Co-60
Cs-137	36900	1100	30.14 years	15-Jul-04	12:00	DKD Cs-137
Mn-54	38700	1100	312.50 days	15-Jul-04	12:00	DKD Mn-54
Y-88	35800	1100	106.6 days	15-Jul-04	12:00	DKD Y-88
Na-22	36900	1100	2.60 years	15-Jul-04	12:00	DKD Na-22
Ra-226	37400	1100	1,600 years	1-Jul-04	12:00	DKD Ra-226
Eu-152	38000	1100	13.33 years	15-Jul-04	12:00	DKD Eu-152
Am-241	36500	1100	432.2 years	1-Jul-04	12:00	DKD Am-241
Co-57	37700	1100	271.80 days	15-Jul-04	12:00	DKD Co-57

Table 2. Efficiency computed and fitted with electronic spreadsheet for GEM 30195 with sources at 15 cm from detector end cap.

Nuclide	Energy (KeV)	Efficiency
Am-241	59.54	0.000613337
Eu-152	121.78	0.003553117
Eu-152	244.7	0.00295213
Eu-152	344.28	0.002491134
Eu-152	367.76	0.002412776
Eu-152	411.3	0.00216846
Eu-152	444	0.002089715
Eu-152	778.9	0.00132318
Eu-152	867.38	0.001252362
Eu-152	964	0.001204472
Eu-152	1112	0.001111782
Eu-152	1212.94	0.001109479
Eu-152	1408.03	0.000931153
Ra-226	1729.6	0.000770708
Ra-226	1764.51	0.000740934
Ra-226	2118.54	0.000589674
Ra-226	2204.12	0.000595507
Ra-226	2447.71	0.000574499

Table 3. Efficiency computed and fitted with electronic spreadsheet for GEM 30195 with sources at 2 cm from detector end cap

Nuclide	Energy (KeV)	Efficiency
Am-241	59.54	0.009084383
Eu-152	121.78	0.045945262
Eu-152	244.7	0.032750254
Eu-152	344.28	0.026325629
Eu-152	367.76	0.023403456
Eu-152	444	0.019739155
Eu-152	778.9	0.012700505
Eu-152	867.38	0.012206212
Eu-152	1212.94	0.00773334
Eu-152	1298.7	0.007228203
Ra-226	1509.19	0.005574393
Ra-226	1661.28	0.004065313
Ra-226	2118.54	0.003298883
Ra-226	2204.12	0.003446851
Ra-226	2293.36	0.003573887
Ra-226	2447.71	0.003721455

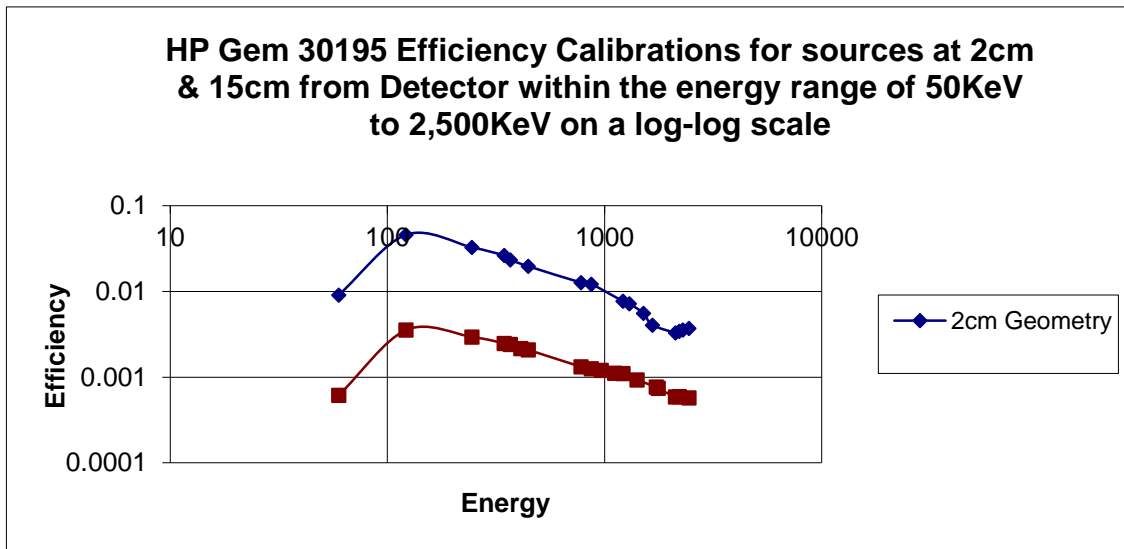


Fig. 7. Efficiency computed and fitted with electronic spreadsheet for GEM 30195.

III. RESULTS AND DISCUSSION

Comparison of the full-energy peak efficiency calibrations using the spreadsheet and k_0 _IAEA software for the two geometries shows good agreement. This is evident from the quality of the two

sets of values of the computed efficiency which is within the range of 10% of each other for the 15cm geometry as shown in Table 4. The higher variations at close geometry of 2cm as seen in Table 5 could be attributed to the disregard of the peak-to-total ratio in the spreadsheet. The peak-to-total ratio which is required for true coincidence effect correction is especially important when the source-detector distance becomes small.

Table 4. Comparison of the full-energy peak efficiency calibration results obtained from spreadsheet and k_0 -IAEA software for 15 cm source-detector distance.

Energy (KeV)	Efficiency		Ratio Spreadsheet/ k_0 -IAEA
	Spreadsheet	k_0 -IAEA	
121.78	3.553E-03	3.415E-03	1.04
244.7	2.952E-03	2.818E-03	1.05
344.28	2.491E-03	2.294E-03	1.09
367.76	2.413E-03	2.207E-03	1.09
411.3	2.168E-03	2.071E-03	1.05
444	2.090E-03	1.983E-03	1.05
778.9	1.323E-03	1.457E-03	0.91
867.38	1.252E-03	1.376E-03	0.91
964	1.204E-03	1.302E-03	0.93
1112	1.112E-03	1.208E-03	0.92
1212.94	1.109E-03	1.155E-03	0.96
1408.03	9.312E-04	1.069E-03	0.87

Table 5. Comparison of the full-energy peak efficiency calibration results obtained from spreadsheet and k_0 -IAEA software for 2 cm source-detector distance.

Energy (KeV)	Efficiency		Ratio Spreadsheet/ k_0 -IAEA
	Spreadsheet	k_0 -IAEA	
121.78	4.595E-02	3.800E-02	1.21
244.7	3.275E-02	2.632E-02	1.24
344.28	2.633E-02	2.032E-02	1.30
444	1.974E-02	1.655E-02	1.19
778.9	1.270E-02	1.057E-02	1.20
867.38	1.221E-02	9.722E-03	1.26
1212.94	7.733E-03	7.483E-03	1.03
1298.7	7.228E-03	7.086E-03	1.02

The k_0 -IAEA software was purposely developed for the implementation of the k_0 -standardization of NAA making it more suitable for spectral analysis. This software has several advantages over the use of electronic spreadsheet in detector efficiency calculations. It has an internal provision for the conversion of the reference geometry efficiency to actual sample geometry efficiency leading to improved accuracy of analytical results due to smaller correction factors. The dynamic nature of the k_0 -IAEA software makes it possible for the analyst to easily jump back and forth between the concentration calculation and peak fit; when troubleshooting is necessary since the program remembers all the peak fitting parameters once it has performed them. The k_0 -IAEA software always corrects for gamma-ray self-absorption in the sample/calibration sources and their recipients as long as the information about the matrix composition and sample/source dimensions have been entered correctly in the software [13]. On the other hand, the electronic spreadsheet is more tedious and labour intensive in carrying out all the necessary calculations [11]. Furthermore, the k_0 -IAEA software package has several other routines in its suite for energy calibration, storing of the detector background spectrum, peak shape calibration and

peak-to-total ratio curves. All these advantages make its analytical results more accurate and dependable.

However, there is still further need for enhancement of the code to make it capable of resolving multiplet or overlapping peaks into individual components for accurate net peak area determination without the user intervention. This will reduce the manual selection of the region of interest (ROI) for some photopeaks before peak area analysis.

IV. CONCLUSIONS

Two approaches (k_0 -IAEA software and electronic spreadsheet) for calculating the full-energy peak efficiency for the high purity germanium detector GEM 30195 have been compared. On the whole, the performance of the two approaches in terms of accuracy and effectiveness are close at higher geometries with less than 10% variation. However, k_0 -IAEA software holds out great prospects with exciting possibilities given its several advantages over the spreadsheet method which have been earlier enumerated in results and discussion. Finally, this study was carried out as part of preliminary studies needed for the implementation of the k_0 -

standardized NAA method for routine elemental analysis in NIRR-1 NAA laboratories. ^[4,11]

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REFERENCES

- [1] Acharya, R.N., Mondal, R.K., Burte, P.P., Nair, A.G.C., Reddy, N.B.Y.,
- [2] Reddy, L.K., Reddy, A.V.R., & Manohar, S.B. 2000. Multi-element analysis of emeralds and associated rocks by k_0 neutron activation analysis. *Applied Radiation and Isotopes*, 53: 981-986.
- [3] Adeleye, M.O., Ibrahim, Y.V., Njinga, R.L., Balogun, G.I., & Jonah, S.A. 2012. Determination of some metal contaminants from industrial effluents in North-West Nigeria using k_0 -NAA Standardization Method. *Advances in Applied Science Research*, 3(2): 678-684.
- [4] Blaauw, M., Bode, P., & De Bruin, M. 1991. An alternative convention describing the (n,γ) -reaction rate suited for use in the k_0 -method of NAA. *Journal of Radioanalytical and Nuclear Chemistry, Articles*, 152(2): 435-445.
- [5] Jonah, S.A., Sadiq, U., Okunade, I.O., & Funtua, I.I. 2009. The use of the k_0 -IAEA program in NIRR-1 NAA laboratory. *Journal of Radioanalytical and Nuclear Chemistry*, Vol. 279, No. 3, 749-755.
- [6] Jonah, S.A., Balogun, G.I., Umar, I.M., & Mayaki, M.C. 2005. Neutron spectrum parameters in irradiation channels of the Nigeria Research Reactor-1 (NIRR-1) for the k_0 -NAA standardization. *Journal of Radioanalytical and Nuclear Chemistry*, Vol. 266, No. 1, 83-88
- [7] Jonah, S.A., Umar, I.M., Oladipo, M.O.A., Balogun, G.I., & Adeyemo, D.J. 2006. Standardization of NIRR-1 irradiation and counting facilities for instrumental neutron activation analysis. *Applied Radiation and Isotopes*, 64, 818-822.
- [8] Njinga R.L., Ibrahim Y.V., Adeleye M.O., & Jonah S.A. 2011. Neutron Flux Stability Measurement of Miniature Neutron Source Research Reactors using 0.1% Au-Al Alloy and Pure Cu wires. *Advances in Applied Science Research*, 2(6): 488-497.
- [9] Limen, N.R., Ibrahim, Y.V., Adeleye, M.O., & Jonah, S.A. 2011. Determination of neutron flux parameters after installation of Cd-lined for implementations of ENAA and FNAA with NIRR-1. *Indian Journal of Scientific Research*, 2(4): 1-9.
- [10] Howard, M. 1991. Principles and practice of spectroscopic calibration. Wiley-Interscience. John Wiley & sons, Inc. New York.
- [11] Digital Gamma-Ray Spectrometer (DSPEC jr 2.0) Hardware User's manual.
- [12] Adeleye, M.O. 2013. Application of the k_0 -NAA Standardization method for the determination of metal contaminants from industrial effluents. Unpublished Ph.D. Thesis. Nuclear Physics. Ahmadu Bello University, Zaria, Nigeria.
- [13] De Corte, F. 2001. The standardization of standardless NAA. *Journal of Radioanalytical and Nuclear Chemistry*, 248(1): 13-20.
- [14] Blaauw M. 2013. Correction for neutron self-shielding and gamma-ray self-absorption in k_0 -IAEA software. *Personal Communication*.
- [15] Blaauw M. 2007. *The k_0 -IAEA program tutorial*. The International Atomic Energy Agency and the Reactor Institute Delft.
- [16] Moon, J. H., Dung, H. M., Kim, S. H., & Chung, Y. S. 2009. Implementation of the k_0 -NAA method by using k_0 -IAEA software and the NAA#3 irradiation hole at the HANARO research reactor. *Journal of Radioanalytical and Nuclear Chemistry*, Vol. 280, No.3, 439-444.
- [17] Rossbach, M., Blaauw, M., Bacchi, M.A., & Xilei, L. 2007. The k_0 -IAEA program. *Journal of radioanalytical and Nuclear Chemistry*, 274(3): 657-662.
- [18] Blaauw, M. 1996. The k_0 -Consistent IRI Gamma-ray Catalogue for Instrumental Neutron Activation Analysis. Interfaculty Reactor Institute, Delft.